



# Integral Inherently Safe Light Water Reactor (I²S-LWR) Concept

Razvoj koncepta integralnog lakovodnog reaktora  
s inherentnim sigurnosnim karakteristikama

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# Outline

- Georgia Institute of Technology (Georgia Tech) and Nuclear and Radiological Engineering (NRE) Program
- Nuclear Power in US and Worldwide
- Development trends:
  - New construction in USA
  - SMRs and “Safety-by-Design”
  - High-temperature high-efficiency reactors
- Next generation LWRs:  
Integral Inherently Safe Light Water Reactor – I<sup>2</sup>S-LWR
- Concluding remarks
- Q&A

# Introductory Remarks on Nuclear Power in USA and Worldwide

# Worldwide use of nuclear power

- 2012: 435 reactors, 370.0 GWe (NN 3/2012)
- 2013: 433 reactors, 371.5 GWe (NN 3/2013)
- About 1/6-th world electricity
- Over 60 new reactors in 14 countries under construction (WNA, 2/2013)
- Major source of electricity in several countries

NUCLEAR POWER UNITS BY NATION

## POWER REACTORS BY TYPE, WORLDWIDE

Reactor Type	# Units	Net MWe	# Units	Net MWe	# Units	Net MWe
	(in operation)		(forthcoming)		(total)	
Pressurized light-water reactors (PWR)	267	246 555.1	89	93 014	356	339 569.1
Boiling light-water reactors (BWR)	84	78 320.6	6	8 056	90	86 376.6
Gas-cooled reactors, all models	17	8 732	1	200	18	8 932
Heavy-water reactors, all models	51	25 610	8	5 112	59	30 722
Graphite-moderated reactors, all models	15	10 219	0	0	15	10 219
Liquid-metal-cooled reactors, all models	1	560	4	1 516	5	2 076
<b>Totals</b>	<b>435</b>	<b>369 996.7</b>	<b>108</b>	<b>107 898</b>	<b>543</b>	<b>477 894.7</b>

March 2012

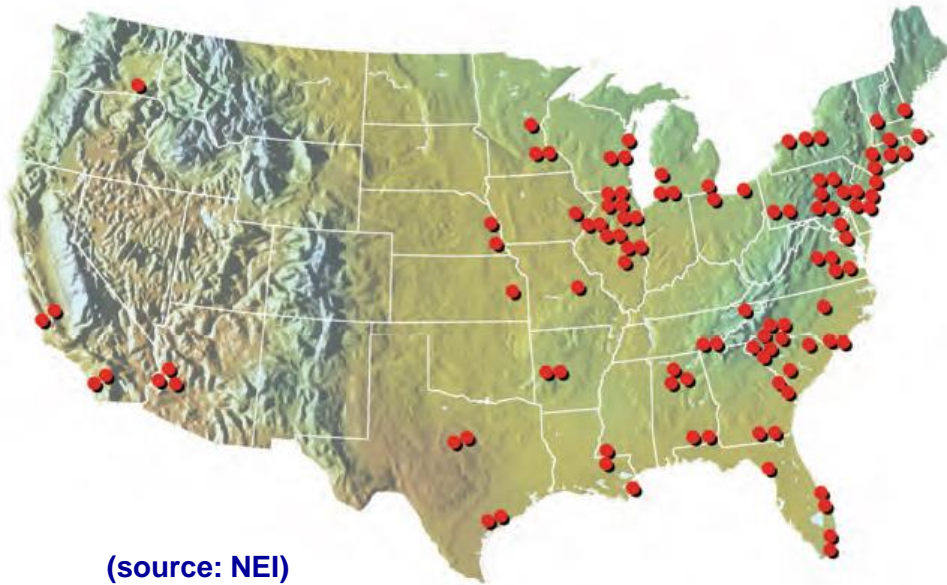
NUCLEAR NEWS

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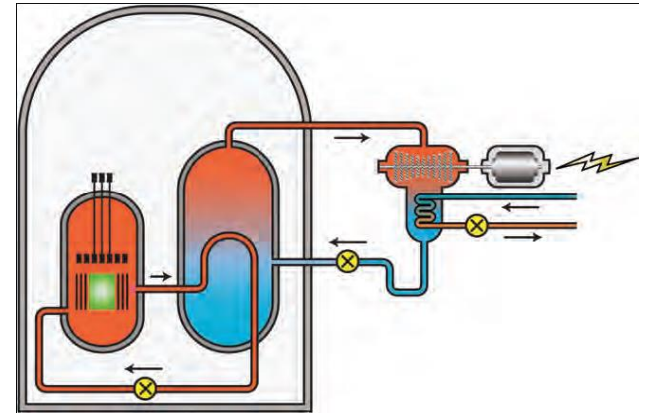
(source: ANS, Nucl. News 3/2012)

# Nuclear power plants in the U.S.

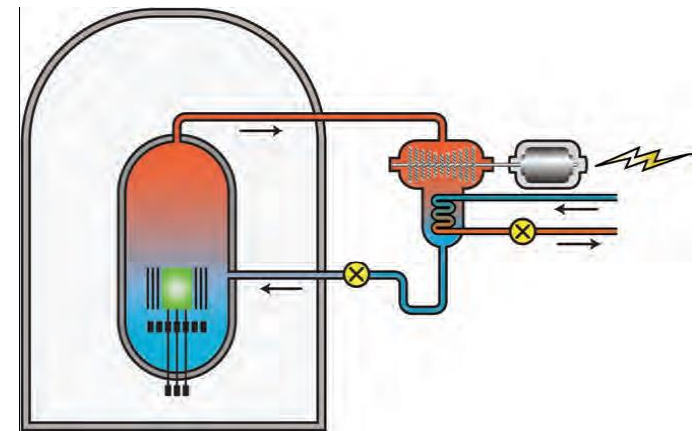
- 100 operating reactors in 31 states
- Close to 20% electricity produced
- 65 PWRs, 35 BWRs
- ~102 GWe



(source: NEI)



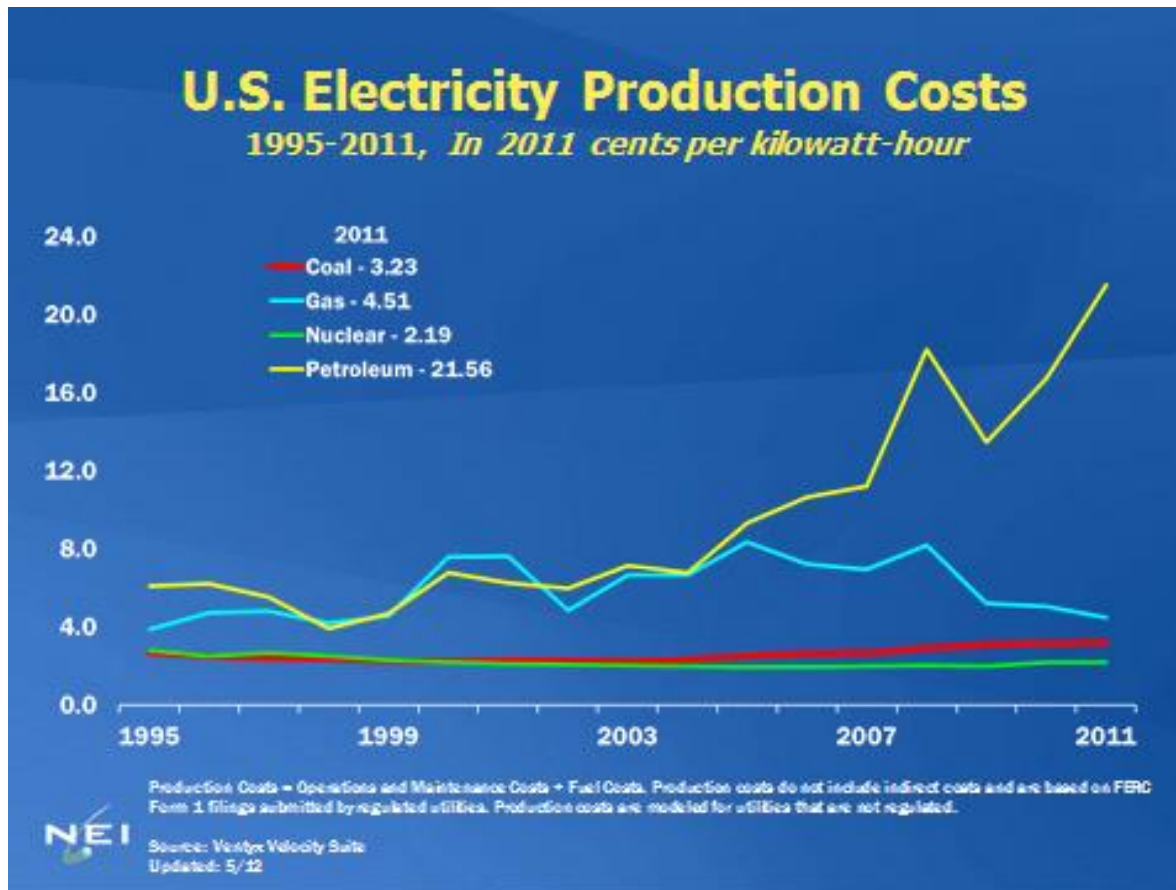
Pressurized Water Reactor (PWR)



Boiling Water Reactor (BWR)

**Nuclear Power Plants –  
Most Expensive Electricity?**

# Energy production cost

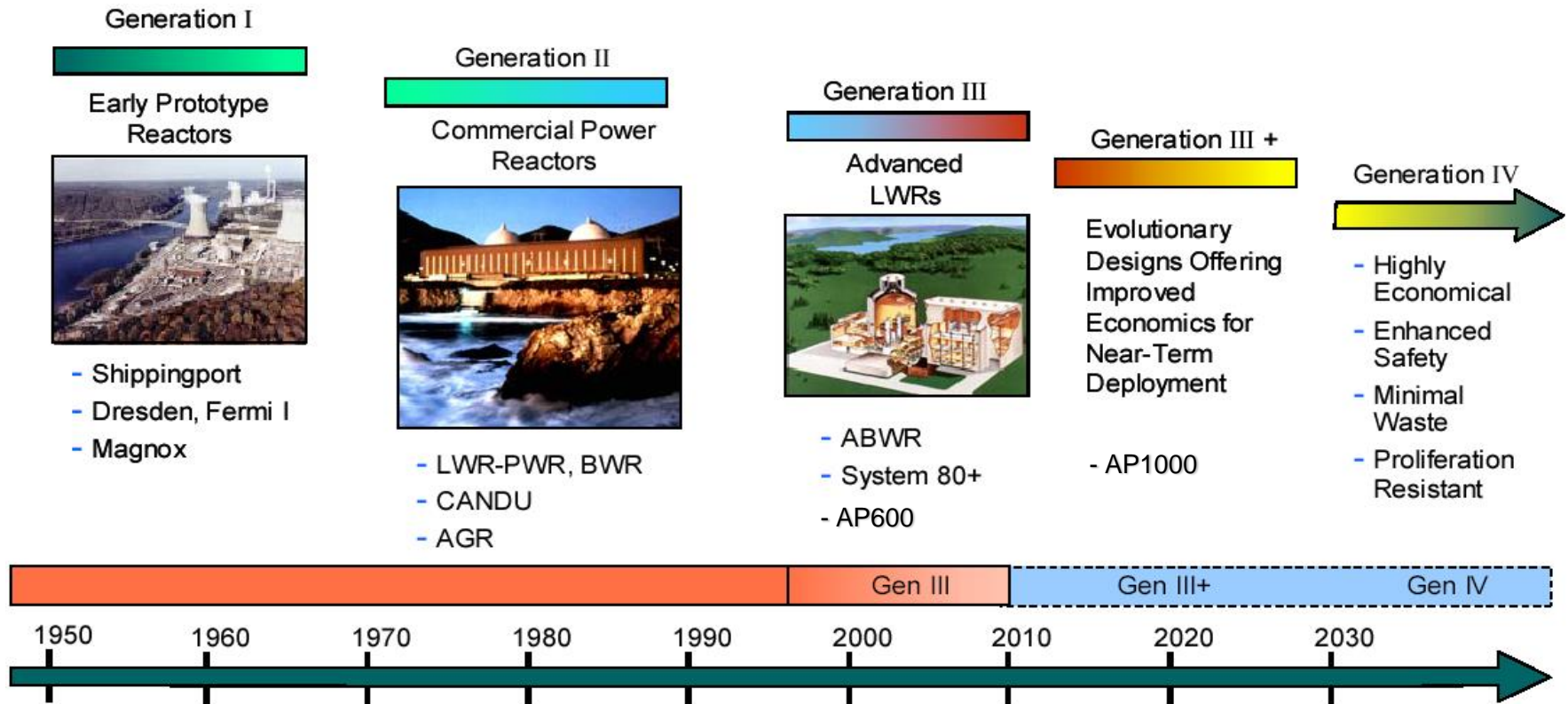


Nuclear power has low electricity production cost (lowest-cost source of electricity over the past 10+ years; it will be initially higher but still competitive for the newly constructed NPPs)

# Nuclear Power – What is New in USA?



# Nuclear power plants – past/present/future



# Nuclear power – What is new in the US?

- New Gen-III+ build in US
- New/advanced designs
  - Gen-IV
  - SMRs
  - Other (**I<sup>2</sup>S-LWR**)
- Impact of The Great East Japan Earthquake (Fukushima)
- Push for “Accident Tolerant Fuel (ATF)” [fuel with enhanced accident tolerance...]
- Nuclear Waste – Long term considerations
  - Yucca Mountain (intended site of deep geological nuclear waste repository)
  - Interim Storage
  - Blue Ribbon Commission on America’s Nuclear Future – Final Report
- New/old fuel cycle options
  - Thorium fuel

# New construction in the U.S.

- 4 new units (AP1000) under construction in USA: 2 in Georgia (Vogtle 3 and 4) and 2 in South Carolina (V.C. Summer 2 and 3); each unit 1,170 MWe

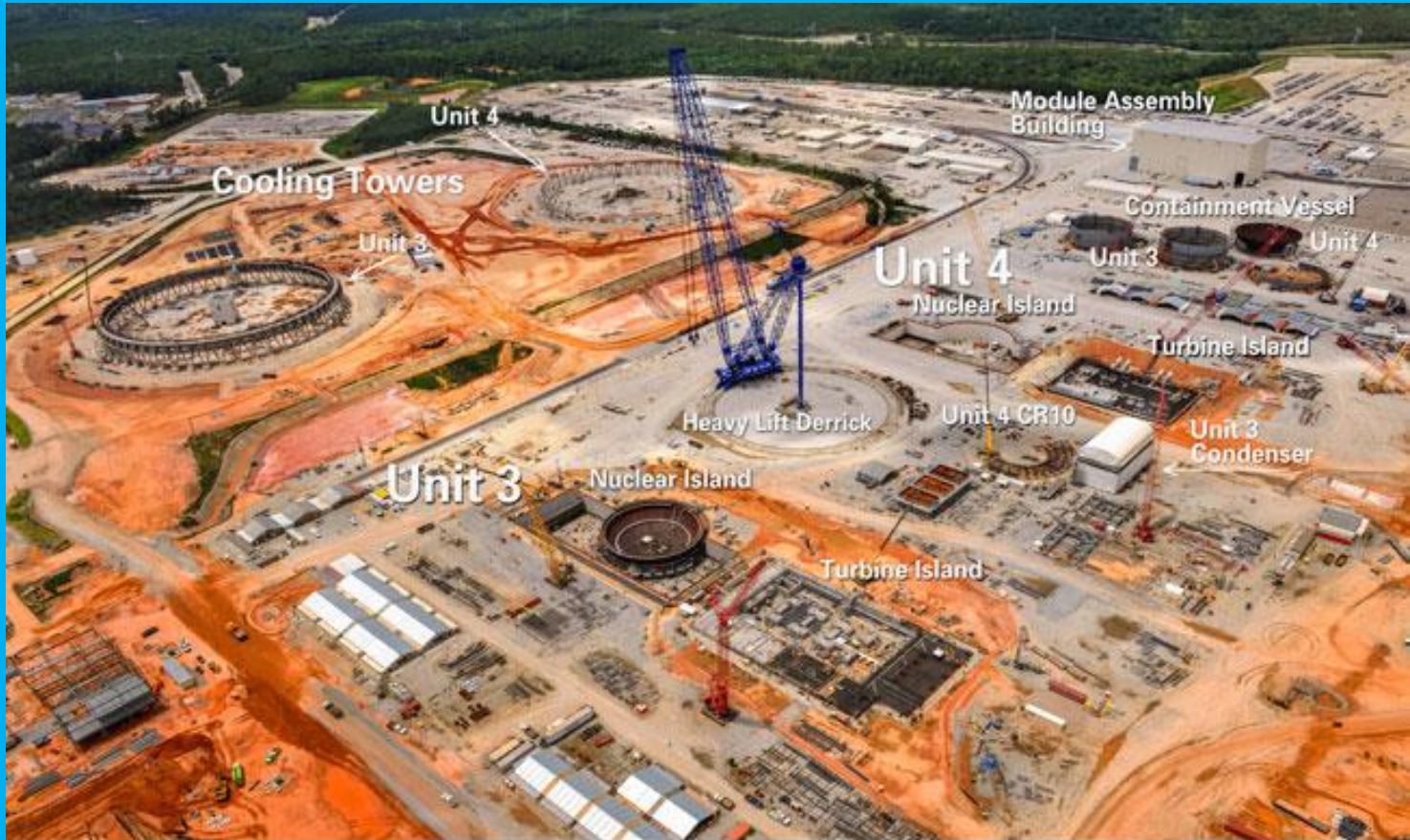


TVA: 2 projects to complete

- Watts Bar 2, PWR (1,180 MWe)
- Bellefonte 1, AL (1,260 MWe), project started in 1974, suspended in 1988, 8/2011 approved, targeting 2018-2020

# New construction in the U.S.

- 2 new units (AP1000) under construction in Georgia, Vogtle 3 and 4 (2x1,170 MWe)
- [Vogtle 3 and 4 Construction Photos](#) [Georgia Power Company.pdf](#)



Plant Vogtle 3 and 4 Construction Site

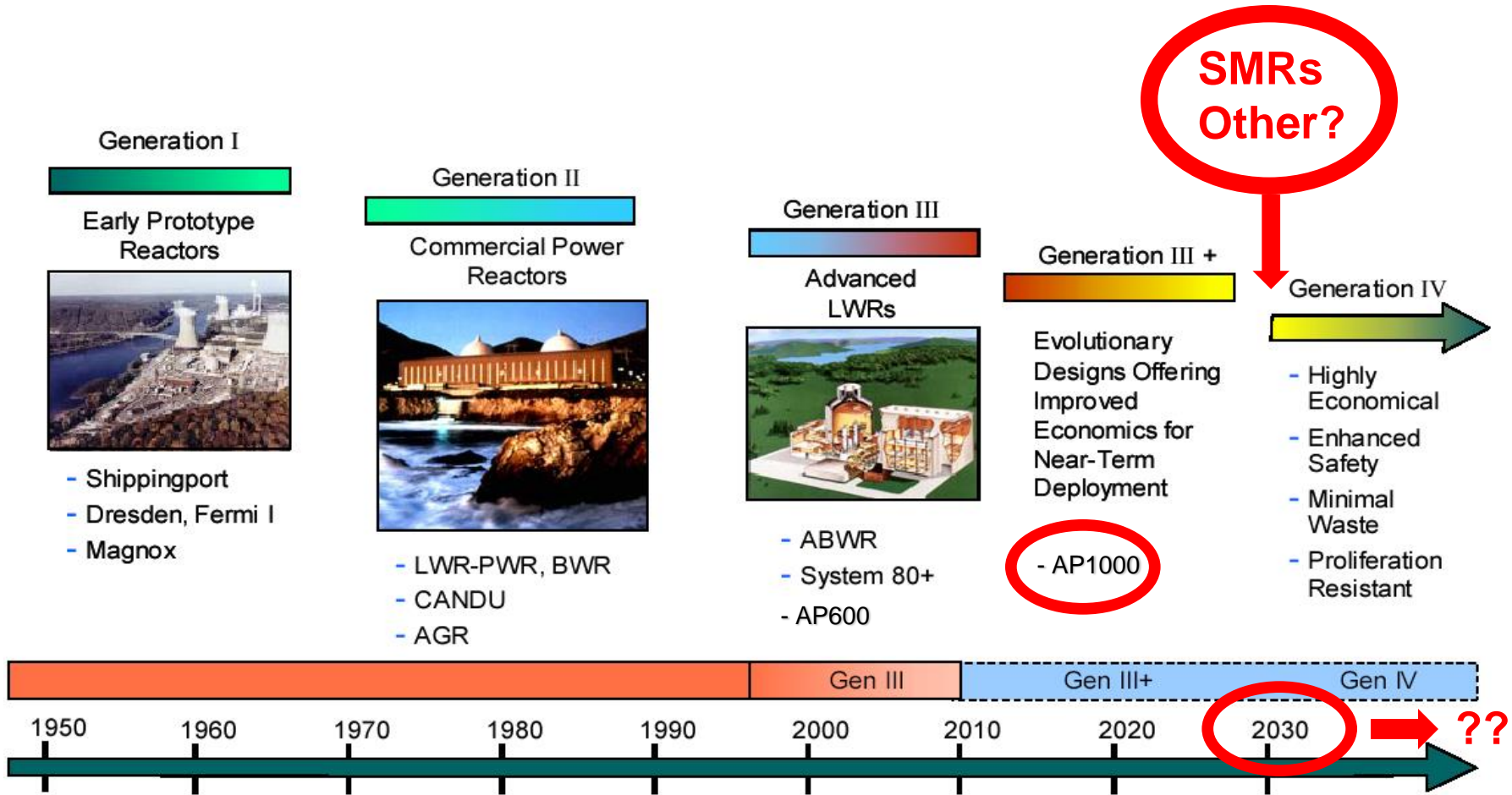
July 2013

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# New/advanced designs

- New/advanced designs
  - “Gen-IV” (Generation IV nuclear power plants) – 6 types
- New/advanced designs pursued at GT NRE
  - SMR (Small Modular Reactors), up to several hundred MWe
    - Reduces the required investment from several billion \$ to <\$1B
    - Extremely high interest recently
  - I<sup>2</sup>S-LWR
    - Inherent safety features
  - Liquid-salt cooled reactors (LSCR), ORNL
    - High temperature, high efficiency, low reject heat, low pressure
  - Hybrid systems
    - high temperature nuclear + energy storage for process heat
    - Nuclear + Renewables (NuRenew)
  - Fusion-fission hybrid (Dr. W. Stacey)

# Nuclear power plants – past/present/future



Fukushima?  
Safety?

# State-of-the-art: Safe enough?

- Gen. III+ Advanced Passively Safe Nuclear Power Plants
- Safety systems operate based on laws of nature (gravity, natural circulation), thus don't require external power, and much less likely to fail than active systems
- Is it safe enough?
- Can it be safer?

Personal perspective:

- ALWRs (and Gen-II LWRs) - extremely safe for all planned/foreseen events
- Inherent safety may (significantly?) improve **response to unforeseen events (Fukushima-type scenario)**



# Inherent safety - examples

## Small power reactors

- Large surface-to-power ratio
- Decay heat removal by conduction

## Integral primary circuit configuration

- All primary circuit components within the reactor vessel
- Eliminates large external piping
- Since it does not exist, cannot break it
- No possibility for LB-LOCA

# SMR

## Small Modular Reactors

# SMRs – Summary and Personal Perspective

- **Attractive safety** (in most cases promoted through integral configuration)
- Emphasis on **modularity and transportability**
- **Power limited to a few hundred MWe**
- **Economic competitiveness “yet to be demonstrated”**
  - “Economy of scale” impact overused as counter-argument (neglects that SMRs may use design features not accessible to large reactor)
  - Licensing cost is a real issue (but it may be overcome)

## Personal perspective

- SMRs can be economical
- SMRs offer a viable option for certain markets
- One size does not fit all; certain markets favor/prefer larger units

Integral Inherently Safe  
Light Water Reactor  
(I<sup>2</sup>S-LWR)

# DOE NEUP IRP

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U.S. DOE – U.S. Department of Energy

NEUP – Nuclear Engineering University Programs

IRP – Integrated Research Project

Only one Integrated Research Project awarded each year for a new reactor concept

# DOE NEUP IRP:

(Nuclear Engineering University Program – Integrated Research Project)

## Integral Inherently Safe Light Water Reactor (I<sup>2</sup>S-LWR) Concept

IRP – DOE’s flagship research program in nuclear engineering for universities (only 1 to 3 awarded annually)

FY13 IRP solicitation requirements:

- Large PWR for US market - economics
- Inherent safety beyond Gen-III+

Multi-institutional, multi-disciplinary team:

### Lead: Georgia Tech

B. Petrovic (PI), NRE/ME/MSE faculty

Ten partnering organizations:

- U. of Michigan, U. of Tennessee, Virginia Tech, U. of Idaho, Morehouse
- National Lab: INL
- Industry: Westinghouse
- Utility: Southern Nuclear
- International: Politecnico di Milano, Italy; U. of Cambridge, UK
- Pending: **University of Zagreb**, Florida Institute of Technology

	Team Members	Co-PIs/Co-Is
<b>Lead</b>	Georgia Tech (GT)	B. Petrovic (PI) F. Rahnema (Co-PI) C. Deo, S. Garimella, P. Singh, G. Sjoden (Co-Is)
<b>Academia</b>	University of Idaho (U-Id)	I. Charit (Co-PI)
	University of Michigan (U-Mich)	A. Manera (Co-PI) T. Downar, J. Lee (Co-Is)
	Morehouse College (MC)	L. Muldrow (Co-PI)
	University of Tennessee (UTK)	B. Upadhyaya, W. Hines (Co-PIs)
	Virginia Tech (VT)	A. Haghghat (Co-PI), Y. Liu (Co-I)
<b>Industry</b>	Westinghouse Electric Company (WEC)	P. Ferroni (Co-PI) F. Franceschini, M. Memmott (Co-Is)
	Southern Nuclear (SNOC)	R. Cocherell (Co-PI)
<b>Nat'l Lab</b>	Idaho National Laboratory (INL)	A. Ougouag (Co-PI), G. Griffith (Co-I)
<b>Int'l</b>	Politecnico di Milano, Milan, Italy (PoliMi)	M. Ricotti (Co-PI)
	University of Cambridge, Cambridge, UK (U-Cambridge)	G. Parks (Co-PI)
<b>Consultant</b>		H. Garkisch

# I<sup>2</sup>S-LWR concept – Top level requirements

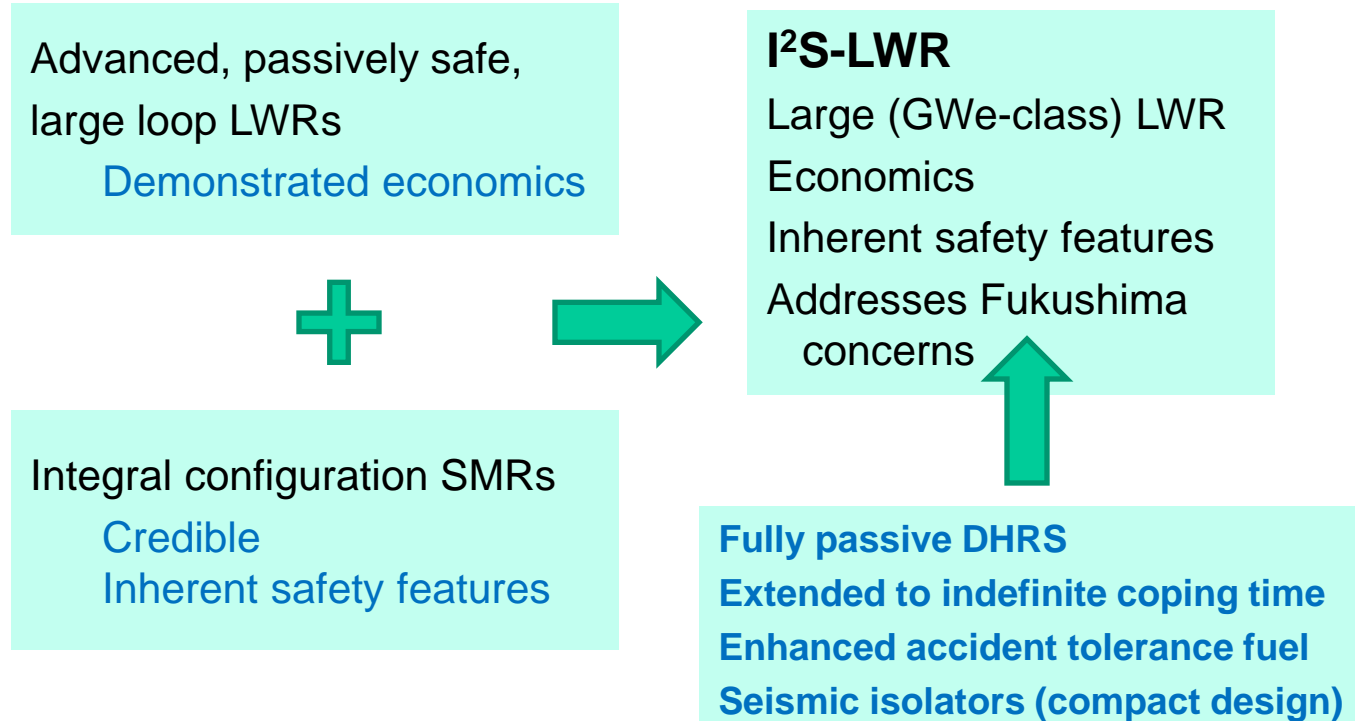
	Requirement	Target	Comment
<b>APPLICATION-DRIVEN REQUIREMENTS</b>			
Power	>910 MWe	1,000 MWe	For markets preferring large plants
Electricity production efficiency	>32%	35%	Competitiveness; reduced reject heat
Design lifetime	60 years	100 years	Competitiveness; economics, sustainability
Reactor pressure vessel	Same size as or smaller than current large PWRs		Manufacturability
<b>FUEL-RELATED REQUIREMENTS</b>			
Fuel/cladding system	Enhanced accident tolerance*		Post-Fukushima considerations
Fuel enrichment	Viable reloading with <5% enriched fuel	Improved fuel cycle with 5-8% enriched fuel	Option to use existing infrastructure for <5% enrichment
Refueling	Multi-batch, refueling interval 12 months or longer	Options for 12-18-24 months refueling	Offer longer cycles when required by utilities
<b>SAFETY AND SECURITY</b>			
Security, safeguards and proliferation resistance	As in current LWRs or better		
Safety indicators	CDF <3x10 <sup>-7</sup> LERF <3x10 <sup>-8</sup>	CDF <1x10 <sup>-7</sup> LERF <1x10 <sup>-8</sup>	Improve safety indicators relative to current Gen-III+ passive plants
Safety philosophy/systems	Inherent safety features Full passive safety High level of passivity		Eliminate accident initiators Eliminated need for offsite power in accidents
Grace period	At least 1-week	Indefinite for high percentage of considered scenarios	Resistance to LOOP and Fukushima-type scenarios
Decay heat removal	Passive system with air as the ultimate heat sink		Resistance to LOOP and Fukushima-type scenarios
Seismic design	Single compact building design	Seismic isolators	Allows siting at many locations
Other natural events	Robust design		Address unforeseen events
Monitoring	Enhanced, in normal and off-normal conditions		Improve normal operation; Address unforeseen events
Spent fuel pool safety	Monitoring Passive cooling		Address Fukushima issues with SFP
Used nuclear fuel management	On-site, for the life of the plant		Remove reliance on repository availability at certain date
<b>DEPLOYMENT REQUIREMENTS</b>			
Economics	Competitive with current LWRs		
Deployment	Near-term: 5% enriched fuel Option: use of oxide fuel	Long-term option: up to 8% enriched silicide fuel	Path to accelerated deployment
Operational flexibility	2-batch and 3-batch, ≥12-month cycle	5% and 8% 12-18-24 months cycle	Diverse market needs
Operational flexibility		Load follow with MSHIM	Reduced effluents (environmental)
D&D	Easily returned to green site		Sustainability and public acceptance

# I<sup>2</sup>S-LWR Concept Overview





# I<sup>2</sup>S-LWR approach to advanced, safe and economical nuclear power plant (extending SMR safety concept to large plants)



# I<sup>2</sup>S-LWR concept - design objectives – what and how?

- **Economics**
  - Large (1 GWe-class)
    - » Compact core
    - » Compact integral HX
- **Inherent safety features**
  - LWR of integral design
- **Fukushima concerns and lessons learnt**
  - Indefinite passive decay heat removal
    - » Natural circulation
    - » Rejection to ambient air
- **Fuel with enhanced accident tolerance**
  - Silicide or nitride (high conductivity)
  - Advanced steel cladding (reduced oxidation at high temperatures)
- **Enhanced seismic resistance**
  - Seismic isolators

# Main challenges (i.e., why not already done?)

Compared to current PWRs:

- Integral configuration → compact core
- Compact core → higher power density core
- Yet, aiming at more accident tolerant fuel
- Requires novel fuel/clad design → require major testing and licensing efforts
  
- Primary HX inside the vessel
- SMR power in such configuration limited to a few hundred MWe
- Requires novel design of several key components
  - Primary HX
  - ...

# I<sup>2</sup>S-LWR approach to advanced, safe and economical nuclear power plant (extending SMR safety concept to large plants)

Advanced, passively safe,  
large loop LWRs  
Demonstrated economics

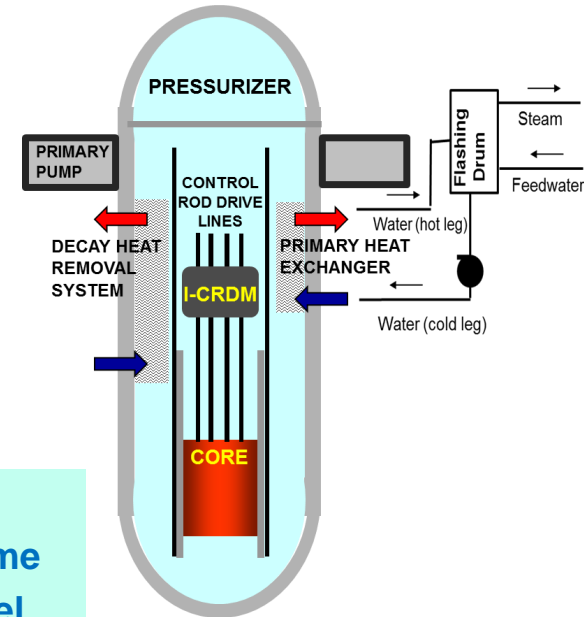


Integral configuration SMRs  
Credible  
Inherent safety features



**I<sup>2</sup>S-LWR**  
Large (GWe-class) LWR  
Economics  
Inherent safety features  
Addresses Fukushima concerns

Fully passive DHRS  
Extended to indefinite coping time  
Enhanced accident tolerance fuel  
Seismic isolators (compact design)



## Key enabling technologies

### Technologies developed for SMRs:

- Integral layout
- Integral primary components

### I<sup>2</sup>S-LWR specific, novel technologies:

- High power density fuel/clad system (silicide fuel)
- High power density (micro-channel type) primary HX (mC-PHX)
- Steam Generation System (mC-PXH + Flashing Drum)

# Additional design features/challenges (a.k.a. the devil is in the “details”)

- Reactor pressure vessel size
- High power density core (flow, vibrations, ...)
- Feasibility of compact HX for nuclear application and this power level (likely feasible, but is it practical/economical?)
- Licensing of a new fuel form/design
- Demonstration of the novel decay heat removal concept
- Integrating/harmonizing all components and systems

# Enabling Technologies

# Key enabling technologies

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## Technologies developed for SMRs

- Integral layout
- Integral primary components

## I<sup>2</sup>S-LWR specific

- High power density fuel/clad system
- High power density primary HX
- Innovative steam generation system (SGG)

# Rationale and selected options for fuel/cladding materials and geometry configuration

- Fuel
  - High-conductivity fuel
  - High HM load
- Cladding
  - Reduced oxidation rate
- Primary choice: Silicide ( $U_3Si_2$  + advanced FeCrAl ODS)



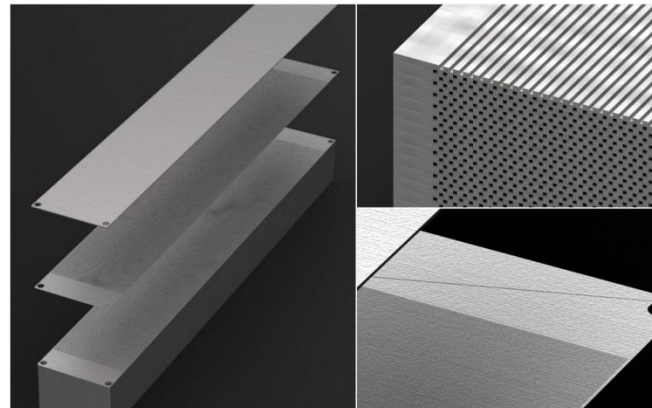
# Fuel Pellet Materials

- Higher U loading of  $U_3Si_2$  vs.  $UO_2$  enables acceptable cycle length at higher specific power and improves fuel management
- Better thermal conductivity lowers T and stored fuel energy
- **Swelling = ?**

Fuel	$U_3Si_2$	$UO_2$
Theoretical density (g/cm <sup>3</sup> )	12.2	10.96
HM Theoretical density (g/cm <sup>3</sup> )	11.3	9.66
Thermal conductivity (unirradiated) (W/m K)	9-20	5-2
	(300-1200°C)	(300-2000°C)
Specific heat J/kg K	230-320	280-440
	(300-1200°C)	(300-2000°C)
Melting point °C	1665	2840

# Innovative steam generation system (SGG)

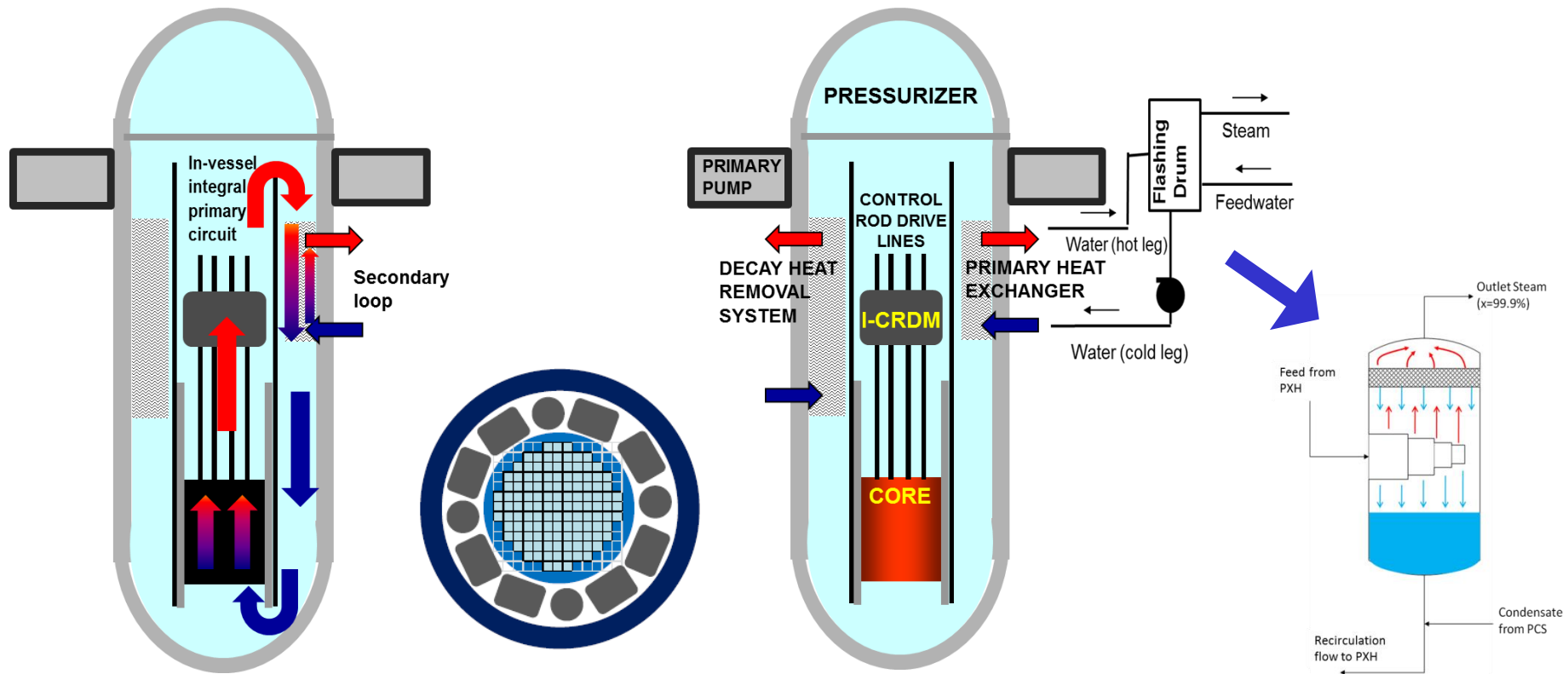
- Integral compact primary HX
  - Microchannel HX
  - High power density
- Combined with external steam drum



# I2S-LWR Layout

# I<sup>2</sup>S-LWR Reactor Layout

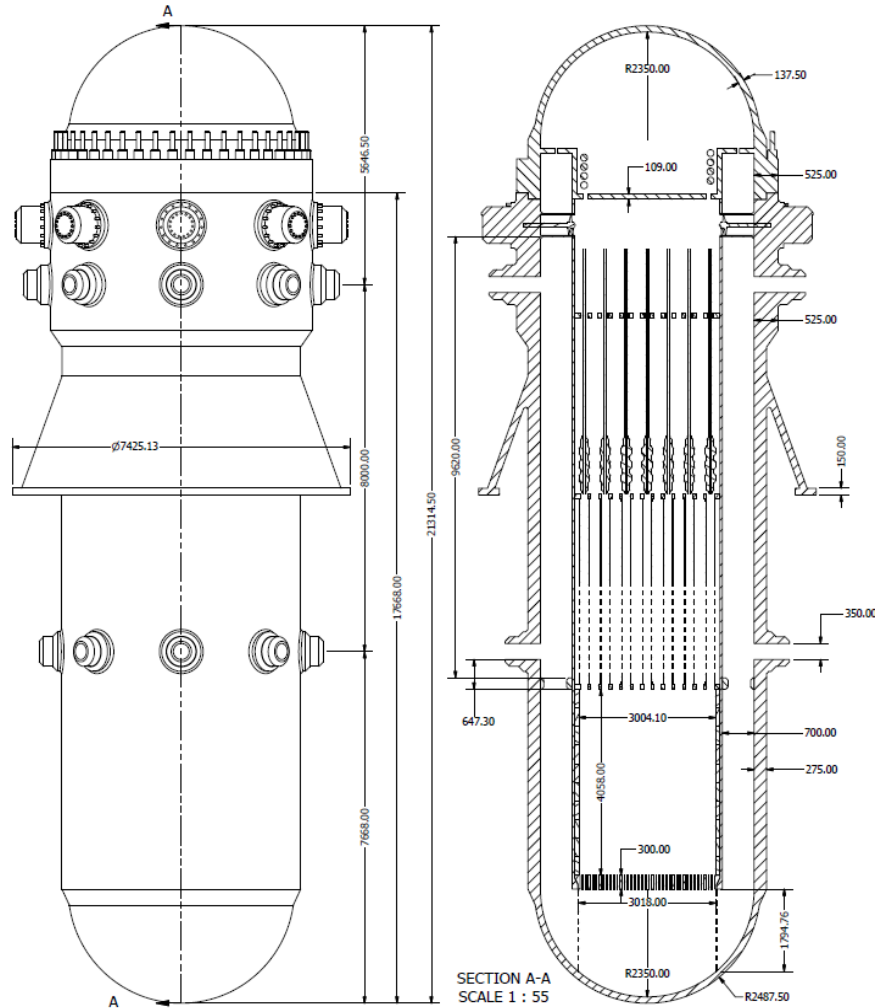
## Integral Configuration



Integral configuration:

- Primary coolant circulates within RPV only
- All primary circuit components (except main pumps) located within the RPV
- 4 SGG subsystems (2 paired modules each): Primary heat exchangers (inside RPV) and flashing drums (outside RPV, inside containment)
- 4 full passive DHRS

# I<sup>2</sup>S-LWR Reactor Layout Integral Configuration




3-D  
printed  
mockup  
1:30 scale



# Examples of a Student Senior Design Project: I<sup>2</sup>S-LWR Integral vessel layout, 3D CAD model

Devised layout, developed 3D CAD model, printed in 1:30 scale (80 cm tall)



## INTEGRAL INHERENTLY SAFE (I<sup>2</sup>S) LIGHT WATER REACTOR

COMPUTER AUTOMATED DESIGN DIMENSIONAL INTEGRATION

BRIAN BARRON • MATT MARCHESE • STERLING OLSON • PAUL ROSE • MICHAEL SAUNDERS • BRIAN SCHWARTZ

Figure Index	Component
1	A Compact Variant Shell-and-Tube HX
2	B 19 x 19 Fuel Assembly
3	C Bottom Core Plate
3	D Neutron Reflector, x10
3	E Upper Alignment Plate
3	F RCC Guide Tube/Upper Support Column
3	G Lower CRDM Plate
3	H Control Rod Drive Mechanism (CRDM)
3	I Upper CRDM Plate
3	J Flow Skirt
4	K Lower Core Barrel
4	L Lower Support Ring
4	M Printed Circuit Heat Exchanger (PCHX)
4	N Decay Heat Removal System
4	O Upper Alignment Plate
4	P Upper Core Barrel
4	Q Pump Impeller
5	R Reactor Pressure Vessel
5	S Pressurizer

**Dimensional Analysis Objectives**

- Generate 3D Computer Automated Design of I<sup>2</sup>S-LWR
- Proof of initial dimensional constraints
- Improvements to integral primary circuit components
- Physical representation of project designed components
- Provide Project baseline dimensional analysis




FIGURE 1: COMPACT VARIANT SHELL-AND-TUBE HX

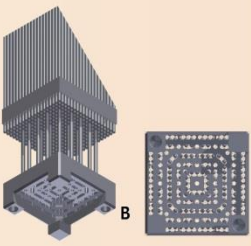


FIGURE 2: 19X19 FUEL ASSEMBLY

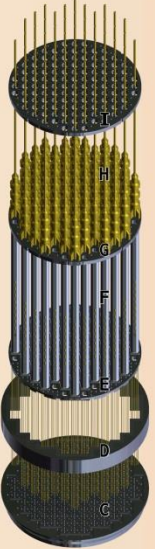


FIGURE 3: RISER ASSEMBLY

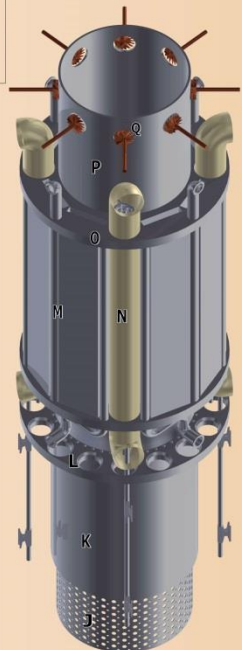


FIGURE 4: DOWNCOMER INTERNALS

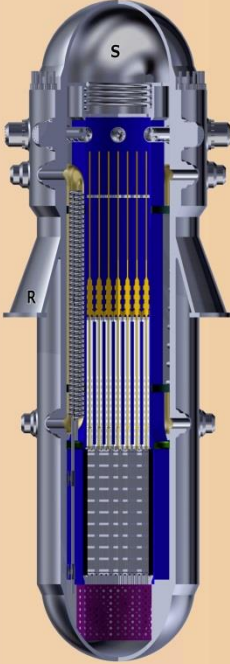
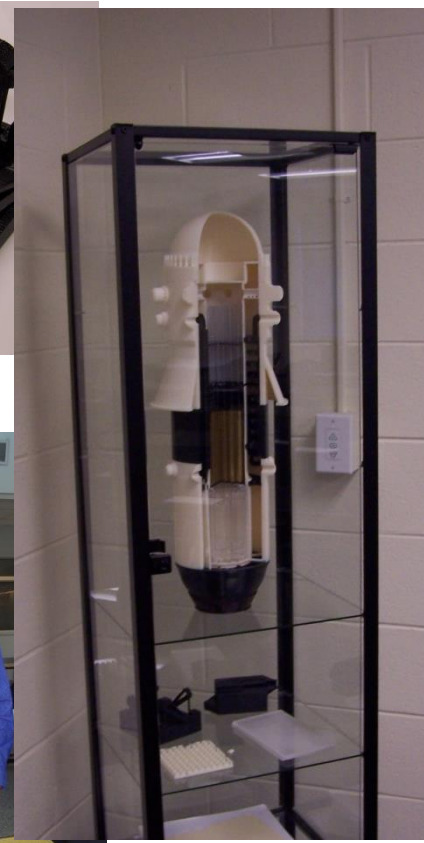
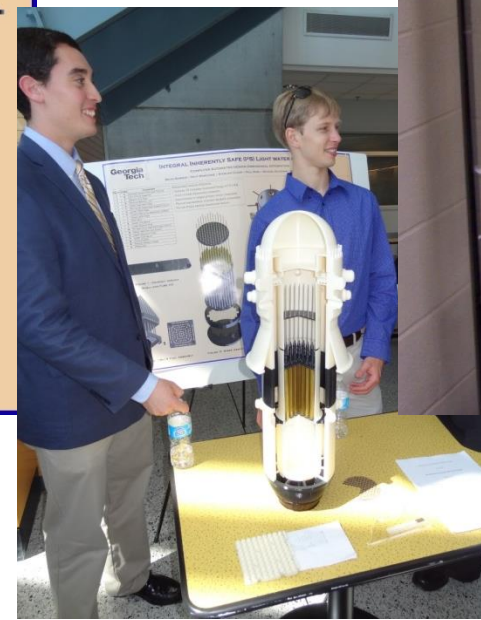
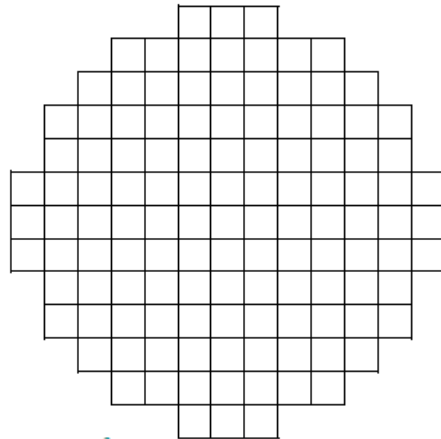
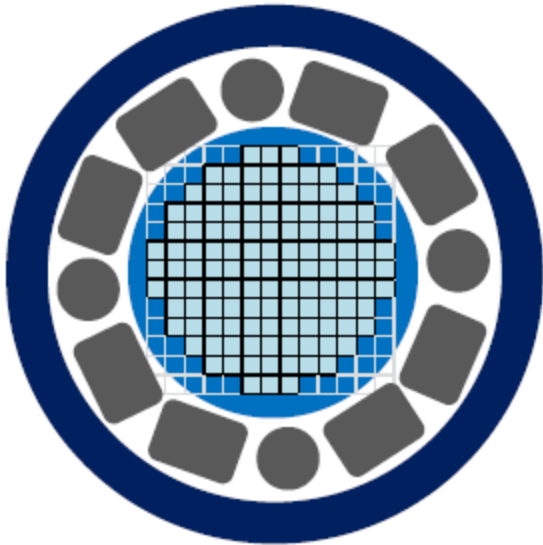


FIGURE 5: REACTOR EXTERNALS



# Core layout and fuel assembly design

- 121 assemblies core configuration, steel radial reflector
- 12 ft active fuel height
- Similar to 2-loop cores but with ~40% higher power rating
- 19x19 assembly, 0.360" fuel rod OD, p/d=1.325



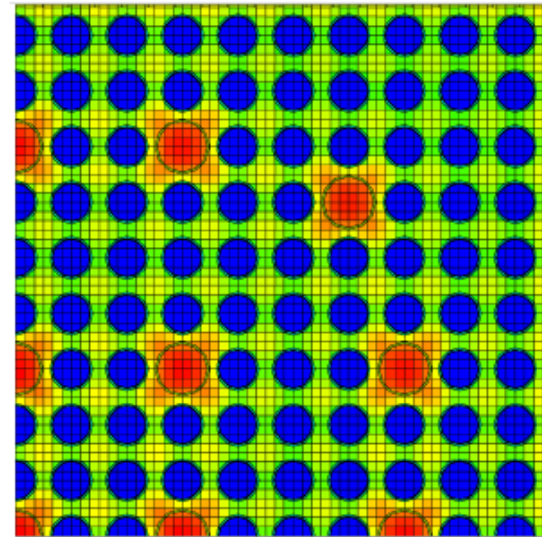
Lattice type	19×19, square
Fuel type	U <sub>3</sub> Si <sub>2</sub>
Cladding material	Advanced SS
Fuel rods per assembly	336
Fuel rod outer diameter (mm)	9.144
Cladding thickness (mm)	0.406
Pellet-clad gap width (mm)	0.152
Pellet outer diameter (mm)	8.026
Pellet inner void diameter (mm)	2.540
Fuel rod pitch (mm)	12.106
Assembly pitch (mm)	231

# Core Concept (5% enrich)

Scoping study on fuel assembly level

- $\text{UO}_2$  (17x17) or  $\text{U}_3\text{Si}_2$  (19x19) fuels
- SS and Zirc-4 Clad
- Evaluated Fuel Cycle Impact on selections
- Included soluble boron and IFBA coatings
- Tentative core design:
  - 19x19 assembly with  $\text{U}_3\text{Si}_2$  fuel
  - 2850 MWth

Geometry Parameter	Value
Rod Diameter [cm]	0.9144
Inner Clad Radius [cm]	0.41656
Fuel Radius [cm]	0.40132
Pitch:Diameter Ratio	1.323
Assembly Pitch [cm]	231
Hydraulic-Diameter [cm]	1.124

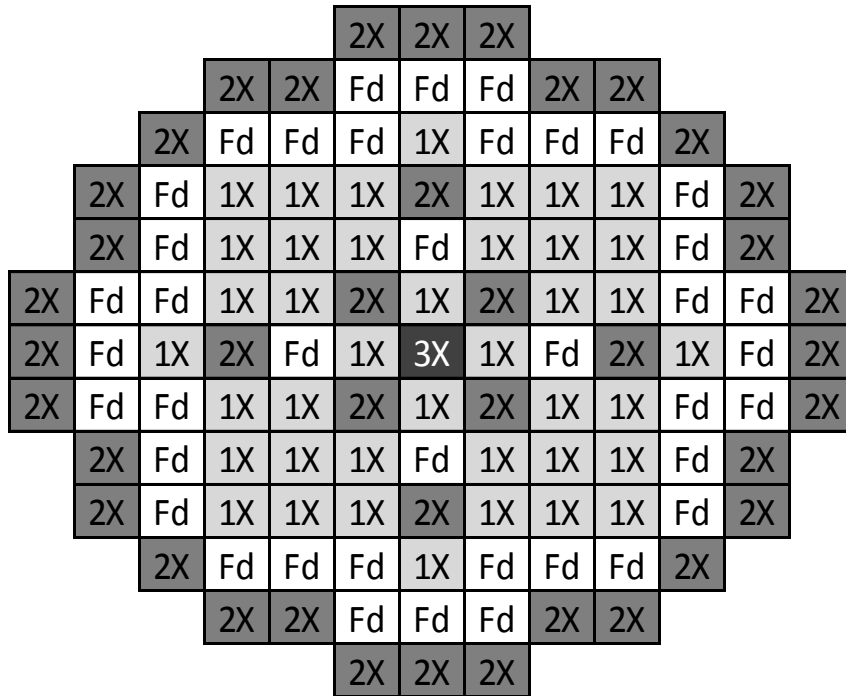


...also 5-8% enrichment analyzed: longer cycle (>2 years)

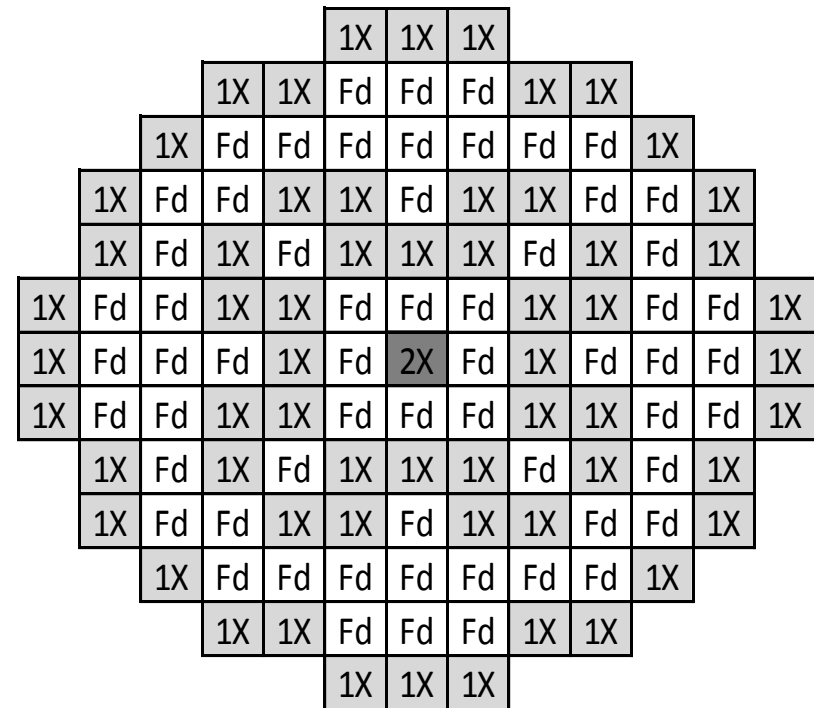


# Fuel Management Schemes for I<sup>2</sup>S-LWR

## 3-batch core with 40 Feed/Reload



## 2-batch core with 60 Feed/Reload



- Full 3-D depletion/reshuffling analysis to equilibrium cycle
- 3-batch /40 Feed-> 3 irradiation cycles before discharge (better fuel use)
- 2-batch /60 Feed -> 2 irradiation cycles before discharge (longer cycle)
- Higher BU fuel assemblies on the periphery (VLLLP)

# Fuel cycle

- Westinghouse evaluated a number of options:
  - » 17x17 and 19x19
  - » 5% and 8%
  - » 12-18-24 months refueling interval
- Viable options:
  - » up to 5% enriched, 12/18-month refueling
  - » up to 8% enriched, 12/18/24-month refueling
- FCC
  - Seems within acceptable range

# Fuel/cladding system

## Economics justification of I<sup>2</sup>S-LWR

New fuel/clad system is enabling technology, aiming to:

- Enable high power density core
- Enable more compact NPP footprint
- Enhance safety

Resulting in economic advantages and disadvantages:

- Neutronics: FCC increased by 15-20%
- More compact NPP layout: capital cost reduced by ?%
- Inherent safety features: some safety systems potentially eliminated, capital cost reduced by ?%

Thus, the trade-off is:

- Reduced capital cost (front-loaded, main portion of COE)
- Increased subsequent FCC

# Safety goals and philosophy

## **MULTIPLE LINES OF DEFENSE**

### ***First line of defense – inherent safety features (eliminate/limit event initiators/precursors)***

- Integral primary circuit eliminates occurrence of LBLOCA/IMLOCA and CR ejection
- Seismic insulator eliminate/limit the impact of seismic events
- Partial burying of containment and underground placement of SFP eliminate/limit external events

### ***Second line of defense - prevention***

- All safety systems are passive with a high degree of passivity and deterministically address DBAs (prevent core damage)

### ***Third line of defense - mitigation***

- Integral configuration with small penetrations limit loss of RPV inventory
- Fuel with enhanced accident tolerance extend grace period
- Passive DHRs extend grace period (potentially indefinitely)
- DPRA-guided design utilizes passive and active systems

### ***Fourth line of defense - protection***

- Containment vessel cooling by air or other medium in natural circulation regime

# Safety goals and philosophy

- As high level of passivity as possible

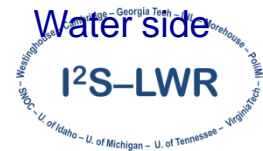
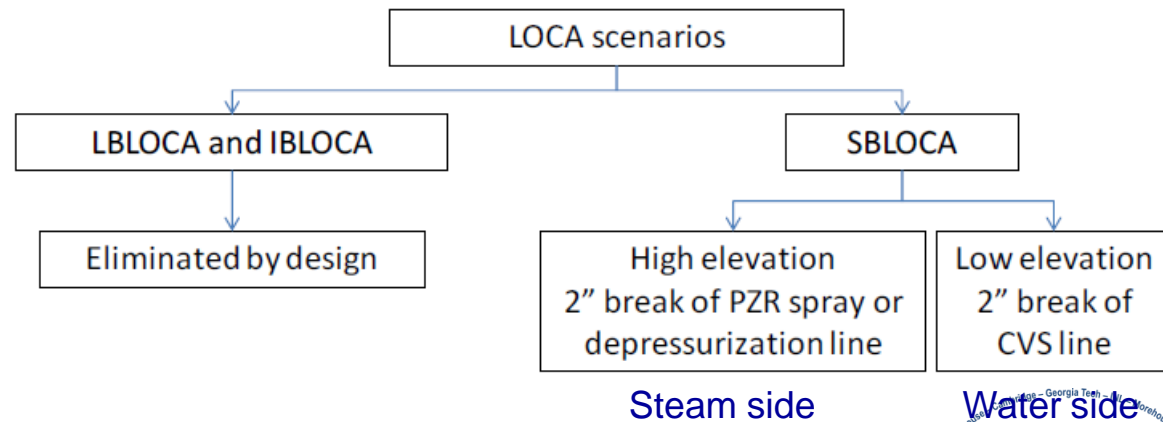
- Eliminate accident initiators as far as achievable

- Limit loss of inventory during LOCAs

Degree of passivity:

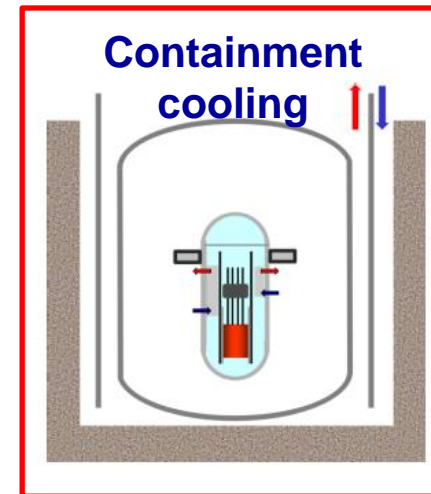
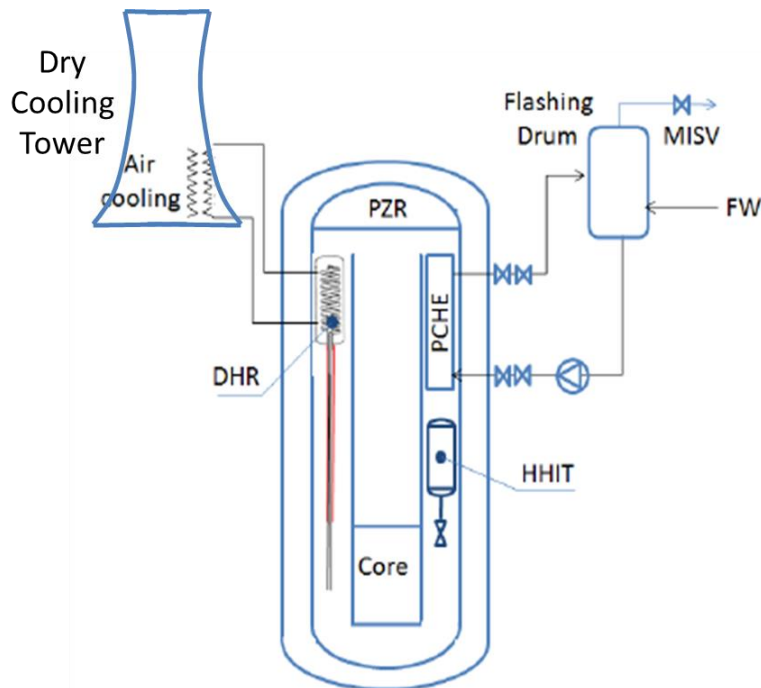
Elements of the passive system	A	B	C	D
Structures (Barriers, pressure proof)	X	X	X	X
Working fluids		X	X	X
Moving mechanical parts			X	X
Stored operating power				X
External activation signal				X

Categories of passive safety systems [IAEA, 1991]



# Safety Systems

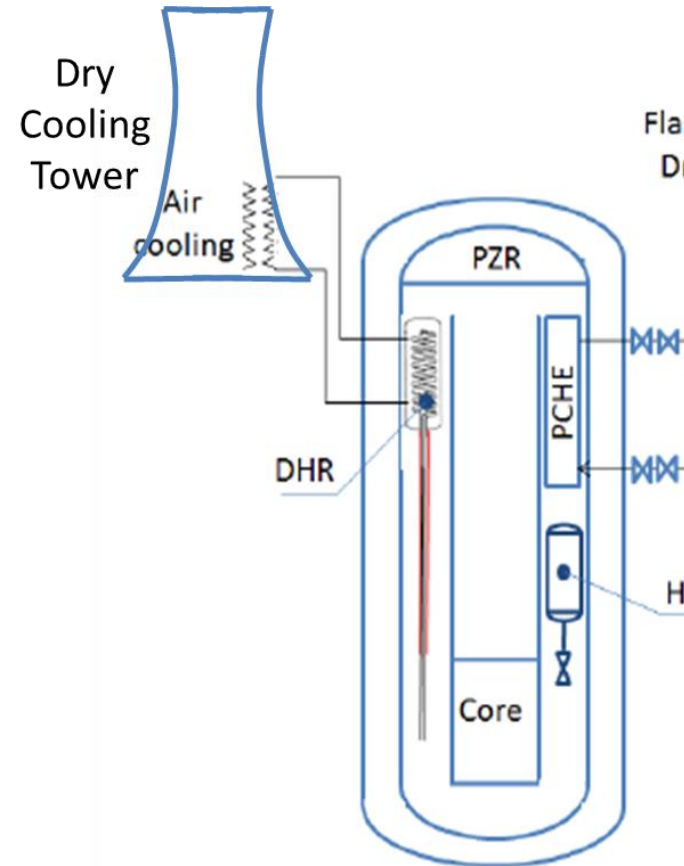
- Passive DHRs (Decay Heat Removal System)
- PHX (mC-HX) as passive heat removal system
- HHIT (High Head Injection Tanks)
- Passive containment cooling



# Passive Decay Heat Removal System

Goal: long term self-sustained decay heat removal capabilities with no need for intervention in case of an accident, including loss of external power

- Passive, natural circulation
- Ultimate heat sink – ambient air
- Four units, sized for 3 of 4
- Target: indefinite heat removal



# Comparison to current large loop PWR

## Similar:

- Core geometry as 2-loop PWR (121 fuel assemblies)
- Fuel assembly similar to 17x17 PWR fuel assembly
- Core internals and control rods
- Secondary and BOP
- Pumps

## Different

### CORE:

- Higher power density (10-30% higher)
- Different fuel form (silicide, ...)
- Enrichment potentially increased (up to 8%)
- Different cladding materials (advanced DS steel)
- Potentially different fuel geometry
- [radial reflector]

### INTEGRAL PRIMARY CIRCUIT:

- Larger reactor vessel (RV)
- PHE (primary heat exchanger) inside RV
- CRDM inside RV
- PZR integrated in RV

### COMPONENTS/SYSTEMS:

- Compact PHE (micro-channel PHE)
- DHRS (decay heat removal system)
  - Natural circulation
  - Ambient air ultimate heat sink
- Seismic isolators

### SAFETY:

- Passive → inherent (features)



# Summary

- New I<sup>2</sup>S-LWR concept aims to extend inherent safety features of SMRs to larger power level reactors
  - Large (~1,000 MWe) PWR
  - Integral configuration
  - Inherent safety features
  - Novel fuel design, components, etc.
- Multi-disciplinary, multi-organization project
- Great opportunity for students to participate in the cutting edge research with involvement of industry and national lab  
(Example: GT - senior design class, 45 students in 2013; ~30 expected in 2014)  
Significant leveraging of DOE funding
- Exciting project – developing potentially the next generation of PWR

# ACKNOWLEDGEMENTS

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Thank you for your attention!